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S. Le Pape \textit{et al.}
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Fusion energy output greater than the kinetic energy of an imploding shell at the National Ignition Facility

S. Le Pape,1 L.F. Berzak Hopkins,1 L. Divol,1 A. Pak,1 E.L. Dewald,1 S. Bhandarkar,1 L. R. Bennettti,1 T. Bunn,1 J. Biener,1 J. Crippen,2 D. Casey,1 D. Edgell,2 D. N. Fittinghoff,3 M. Gatu-Johnson,4 C. Goyon,1 S. Haam,1 R. Hatarik,1 M. Havre,2 D. D-M. Ho,1 N. Izumi,1 J. Jaquez,2 S. F. Khan,1 G. A. Kyralla,3 T. Ma,1 A. J. Mackinnon,1 A. G. MacPhee,1 B. J. MacGowan,1 N. B. Meenan,1 J. Milovich,1 M. Millot,1 P. Michel,1 S. R. Nagel,1 A. Nikroo,1 P. Patel,1 J. Ralph,1 J. S. Ross,1 N. G. Rice,2 D. Strozzi,1 M. Stadermann,1 P. Volegov,3 C. Yeamans,3 C. Weber,1 C. Wild,8 D. Callahan,1 and O. A. Hurricane1

1Lawrence Livermore National Laboratory, Livermore, CA 94550, USA
2General Atomics, San Diego, California 92186, USA
3Laboratory for Laser Energetics, University of Rochester, Rochester, New York 14636, USA
4Plasma Science and Fusion Center, Massachusetts Institute of Technology, Cambridge, Massachusetts 02139
5Los Alamos national Laboratory, Los Alamos, New Mexico 87545, USA
6Diamond Materials Gmbh, Freiburg, Germany
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A series of cryogenic, layered Deuterium-Tritium implosions (DT) have produced, for the first time, fusion energy output twice the peak kinetic energy of the imploding shell. These experiments at the National Ignition Facility (NIF) utilized High Density Carbon (HDC) ablators with a 3-shock laser pulse (1.5 MJ in 7.5 ns) to irradiate low gas-filled (0.3 mg/cc of helium) bare depleted uranium hohlraums, resulting in a peak hohlraum radiative temperature $\sim$ 290 eV. The imploding shell, composed of the non ablated HDC and the DT cryogenic layer, is thus driven to velocity on the order of 380 km/s resulting in a peak kinetic energy of $\sim$21 kJ, which once stagnated produced a total DT neutron yield of $1.9 \times 10^{16}$ (shot N170827) corresponding to an output fusion energy of 54 kJ. Time dependent low mode asymmetries that limited further progress of implosions have now been controlled, leading to an increased compression of the hot spot. It resulted in hot spot areal density ($\rho \sim 0.3$ g/cm$^2$) and stagnation pressure ($\sim$360 Gbar) never before achieved in a laboratory experiment.

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The potential of nuclear fusion as an efficient source of energy was identified decades ago [1]. However, harnessing fusion for energy production has proven to be a difficult task. Throughout the world, billions of dollars are invested in experimental facilities and programs with the goal of demonstrating ignition - the point at which the amount of energy produced via fusion reactions is equal to or greater than the energy supplied to initiate the process[2]. At Lawrence Livermore National Laboratory, the indirect drive approach for Inertial Confinement Fusion (ICF) is pursued at the National Ignition Facility (NIF) [3]. Most ICF work on the NIF is based on the hot spot ignition concept, where the kinetic energy of an imploding shell is converted, upon stagnation, to internal energy in a central hot spot. Fusion is initiated in the hot spot, and a thermonuclear burn front propagates radially outward into the main fuel producing high gain, if the main fuel is of sufficiently high areal density. Ignition is only achieved when self-heating of the hot spot occurs: 3.5 MeV $\alpha$ particles produced by the D+T fusion reactions transfer their energy to the central hot spot to compensate for bremsstrahlung, conduction, and any other energy losses. The theoretical fusion yield can be in the megajoule range, exceeding by a factor of 1000 the kinetic energy supplied to the DT fuel by the implosion alone. While megajoule fusion yields are the goal of the ICF program, reaching that stage requires achieving distinct steps in target gain and yield, each representative of the understanding and resolution of key issues.

The High foot design, in reducing the implosion vulnerability to hydrodynamic instabilities plaguing low adiabat implosions, achieved a net fuel gain (as defined in Ref. 28) in a hot spot dominated by alpha heating [4]. Nevertheless the high foot implosions plateaued near 26 kJ of fusion yield, hot spot spot areal density $< 0.2 g/cm^2$ and 250 Gbar of stagnation pressure. Detailed analysis have shown that symmetry swings were in part responsible for the implosion degradation [5], and the high apparent ion temperature measured [6],[4].

The HDC experiments we report on, by controlling low mode asymmetry through the laser history, achieved for the first time a fusion yield (54 kJ) twice the kinetic energy of the imploding shell ($E_K \sim 21kJ \pm 5kJ$).

$$E_K = \frac{1}{2}(M_{HDC} + M_{DT})V_{imp}^2$$  \hspace{1cm} (1)

Where the mass of the DT cryogenic layer $M_{DT}$ is 0.13 mg, the mass of the non ablated HDC $M_{HDC}$ is 0.13 mg $\pm$0.03, and the maximum implosion velocity $V_{imp}$ is 380 km/cm $\pm$ 30.
At similar adiabat to the High Foot campaign, the HDC approach of minimizing low mode asymmetry of the X-ray drive throughout the implosion history [7],[8] led to a hot spot $\rho r$ ($\sim$0.3 g/cm$^2$), high enough to stop most ($\sim$85%) of the $\alpha$ particles. The hot spot areal density is now high enough (see Fig.4) to sustain self heating once the confinement time is increased. The energy deposited by the $\alpha$ particle in the hot spot is now $\sim$10 kJ, more than twice the $\alpha$ deposited energy of any previous experiment [4]. Furthermore, the stagnation pressure of the hot spot ($\sim$360 Gbar) is now higher than the pressure of the solar core [9].

Consequently, the conditions achieved in the hot spot now enable access to a range of nuclear and astrophysical regimes. The density, temperature and pressure of the hot spot are the closest on earth to conditions in the sun[10], [11], and the neutron density ($>10^{23}$neutrons/cc) is now relevant for nucleosynthesis studies (such as the s-process), which have traditionally been in need of an intense, laboratory-based neutron source [12],[13].

To reach these hot spot conditions, the shell has to implode nearly spherically at all time[14]. The HDC shell sits at the center of a high-Z cylinder (‘hohlraum’) irradiated by the 192 NIF laser beams, the symmetry of the X-ray radiation bath resulting from the interaction of the laser beams with the hohlraum walls dictates the symmetry of the imploding shell. The use of HDC ablators [15],[16][17] has enabled us to lower X-ray drive asymmetries. Thin (70µm) HDC ablators permit the usage of shorter laser pulses (<9ns), facilitating symmetry control as the hohlraum fills over time with expanding high Z plasma from the hohlraum wall (bare depleted uranium in this paper).

The implosion symmetry is controlled throughout the laser drive history by adjusting the relative power balance between the inner and outer laser cones. Following the methodology described in [7] using a 5.75m diameter hohlraum and a 844µm inner radius (scale 5.75) HDC shell, the implosion symmetry was measured and optimized at the larger 6.2 scale (figure 1). Figure 2 provides an overall quantitative view of symmetry measurements obtained at different times and convergences using multiple experimental platforms (2 axis keyhole[18], 2D radiograph of the infight compressed shell[19], low convergence gas-filled capsule ("Symcap")[20] and high convergence cryogenic DT layer [21]). At all times along the shell trajectory, symmetry is controlled to better than $\pm$10 µm; hot spot at bang time is within 6 µm of round.

Following this series of symmetry experiments, two cryogenic DT layered experiments (shots N170601 and N170827) were carried out to test fusion performance at high convergence. The hohlraum was driven to $\sim$290 eV radiation temperature by 1.5 and 1.7 MJ of laser energy at 450 TW peak power (figure 1-c). Figure 3 shows an equatorial and a polar image of the measured primary neutron emission (12-15 MeV) as well as the reconstructed neutron production volume at bang time [22] for N170601 (N170827 has a similar hot spot shape and volume). The neutron volume is slightly ellipsoidal, with a measured P2 of -6 ±2 µm, (as fit with the 2$^{nd}$ Legendre moment, P2).
Table I summarizes the results of these two shots (N170601 and N170827) and for comparison, one of the best performers of the High Foot Campaign (shot N140304) [23]. The High Foot campaign is a high adiabat campaign based on a CH ablator, that reached the α heating regime by reducing the impact of hydrodynamic instabilities on the hot spot compression [4], [24]. Neutron yield, Down Scatter Ratio (DSR) and DT ion temperature shown in Table I are directly measured by Neutron Time Of Flight (NTOF) detectors [25]. Quantities such as hot spot ρr, hot spot energy, stagnation pressure and α deposited energy are inferred from experimental observables using a "hot spot" model described in [26].

The total neutron yield is derived from the measured primary neutron yield and DSR using the relation yield_{total} = yield_{13-15MeV} \times 4^{DSR} [27]. The hot spot density is then derived from the measured yield, burn width, and neutron volume using the relation [28].

\[ Y = <\sigma V > \tau V_{hs} N_D N_T \]  

where \(<\sigma V >\) is the equimolar DT reactivity, which is a function of the ion temperature. \(V_{hs}\) is the volume of neutron emission, which is calculated using the equatorial and polar neutron images, and \(\tau\) is the neutron burn width measured by the Gamma Reaction History (GRH) detector [29]. The hot spot stagnation pressure is inferred from the hot spot density and ion temperature measured by the NTOF detectors using the relation \(P_{stag} = \left[ (Z + 1) / \bar{A} m_p \right] \rho T_{ion}\), where \(Z=1\) for D-T, \(\bar{A}=2.5\) is the average atomic mass number. This leads to the determination of the hot spot energy using the relation \(E_{hs} = 3/2P_{stag} V_{hs}\).

To infer the energy deposited by α particles, the fraction of α energy deposited \(f_\alpha\), assuming a spherical hot spot, is first calculated. It is a function of the hot spot \(\rho r\) and ion temperature [30].

\[ f_\alpha = 1 - \frac{1}{4((\rho r)_{hs}/\rho \lambda_\alpha)} + \frac{1}{160((\rho r)_{hs}/\rho \lambda_\alpha)^3} \]  

where the α particle stopping range is

\[ \rho \lambda_\alpha = \frac{0.025 T_e^{5/4}}{1 + 0.0082 T_e^{5/4}}. \]  

where \(T_e\) is the electron temperature, assuming \(T_e=Ti\), in base units of centimeters, grams, and kiloelectronvolts. The energy partition in a D-T fusion event is 80% to a 14.1 MeV neutron and 20% to a 3.5 MeV α particle. Therefore, the energy deposited by an α particle in the hot spot can be derived from the fraction of α deposited energy and the total fusion yield.

A key achievement of the High Foot campaign in 2014 was to achieve a fuel gain of unity: more fusion energy was produced in the hot spot than energy was delivered to the DT fuel. This demonstration required a detailed estimate of the energy balance during stagnation. Here, we have improved enough the implosion to report a net gain of 2 between the fusion energy produced and the maximum kinetic energy of the implosion shell, including both the DT fuel and the remaining ablator, at peak implosion velocity, when the hohlraum does not accelerate the capsule anymore. This can be thought as the fusion gain of an isolated system (the free-falling imploding shell) and reporting the gain does not have to rely on internal mechanics. The shell velocity is measured using the 2DconA platform (N170419 shot), the position of the shell is recorded as a function of time on a X-ray gated imager. The fraction of non-ablated carbon at peak velocity is calculated by HYDRA and consistent with in-flight X-ray radiography. For N170827, the fusion energy (54 kJ) is more than twice the maximum kinetic energy of the implosion shell (21 kJ, see before for details), while for one of the high performing HF implosion, N140304, the output energy is about equal to the maximum kinetic energy of the implosion shell (see table I).

As a result of reducing the symmetry swing in the X-ray drive, the improved compression increased the hot spot \(\rho r\) by more than 50% and stagnation pressure by more than 60%. In addition to twice the fusion yield, the energy deposited by the α particle in the hot spot increased from \(\sim 3.4\) kJ to \(\sim 9.7\) kJ. For N170827, with a hot spot \(\rho r\) of 0.3 g/cm² and a \(T_{ion}\) of 4.7 keV, \(f_\alpha \sim 0.87\) which implies that the bulk of the α particles are stopped in the hot spot.

A static, isobaric hot spot model [28] can be used to estimate the energy balance in the hot spot at peak compression. The self heating condition for an isobaric hot spot can be written:

\[ (A_\alpha <\sigma v > f_\alpha - A_b T^{\alpha/2}) (\rho r)^2 - \frac{3c_e A_b T^{7/2}}{ln \Lambda} > 0 \]  

FIG. 3. shot N170601 a) Polar neutron image. b) Equatorial neutron image. c) Three-dimensional reconstructed neutron volume of the hot spot.
TABLE I. Summary of experimental data from cryogenic DT layer implosions at 6.20 scale (N170601, N170827) and for a high performance High Foot experiment (N140304)

<table>
<thead>
<tr>
<th></th>
<th>N170601 data</th>
<th>N170827 data</th>
<th>N140304 data</th>
</tr>
</thead>
<tbody>
<tr>
<td>Total neutron yield</td>
<td>1.7e16 ±2.4e14</td>
<td>1.9e16 ±3e14</td>
<td>9.3e15 ±1.7e14</td>
</tr>
<tr>
<td>fusion yield (kJ)</td>
<td>48</td>
<td>53</td>
<td>26</td>
</tr>
<tr>
<td>DT T (keV)</td>
<td>4.5 ±0.12</td>
<td>4.5 ±0.15</td>
<td>5.5 ±0.12</td>
</tr>
<tr>
<td>DSR (%)</td>
<td>3.27 ±0.2</td>
<td>3.24 ±0.2</td>
<td>3.4 ±0.2</td>
</tr>
<tr>
<td>Velocity (km/s)</td>
<td>381</td>
<td>395</td>
<td>380</td>
</tr>
<tr>
<td>P$\text{stag}$ (Gbar)</td>
<td>320 ±40</td>
<td>360 ±45</td>
<td>222 ±15</td>
</tr>
<tr>
<td>Nuclear Burn width (ps)</td>
<td>160 ±30</td>
<td>154 ±30</td>
<td>163 ±30</td>
</tr>
<tr>
<td>hot spot $\rho s r$ (g/cm$^2$)</td>
<td>0.26 ±0.032</td>
<td>0.30 ±0.034</td>
<td>0.13 ±0.021</td>
</tr>
<tr>
<td>$f_\alpha$ deposited fraction $f_\alpha$</td>
<td>0.81</td>
<td>0.87</td>
<td>0.58</td>
</tr>
<tr>
<td>hot spot energy (kJ)</td>
<td>4.3 ±1.17</td>
<td>4.7 ±1.7</td>
<td>3.6 ±1.03</td>
</tr>
<tr>
<td>Shell max kinetic energy (kJ)</td>
<td>22±5</td>
<td>21±5</td>
<td>25±7</td>
</tr>
<tr>
<td>Alpha deposited energy (kJ)</td>
<td>8±1.36</td>
<td>9.3±1.6</td>
<td>3.3±0.58</td>
</tr>
</tbody>
</table>

Where $< \sigma v >$ is the DT reactivity, $f_\alpha$ is the fraction of $\alpha$ energy deposited, $A_\alpha$=8.1 $10^{40}$ erg/g$^2$, $A_b$=3.5 $10^{23}$ erg/g$^{-2}$cm$^3$s$^{-1}$, $T$ is the ion temperature, $\rho r$ is the areal density in g/cm$^2$, $ln\Lambda$ is equal to 3.7 in our hot spot conditions, $c_\epsilon=1$, $A_\epsilon$=9.5 $10^{19}$ erg/keV$^{-7/2}$cm$^{-1}$s$^{-1}$. The first term in equation (5) is the deposited fusion power; the second term is the bremsstrahlung emission power density, and the thermal conduction power density is the third term.

At the conditions achieved on N170827, Bremsstrahlung and electron conduction losses are still dominating the alpha deposited energy. We can estimate from N170827 conditions (at constant $\rho r$ and adiabat) what it would take to reach equilibrium and the onset of the burning plasma regime. At constant $\rho r$, the $\alpha$ deposited energy scales like $< \sigma v >$, which roughly scales like $T^4_{\text{ion}}$ [28], the equilibrium is thus reached for a ion temperature of 4.8 keV, corresponding to a neutron yield of 2.4x10$^{16}$. The two solid lines shown on figure 4 are the yield extrapolation at constant $\rho r$ for the best High Foot and HDC shots based on the DT cross section dependence with temperature. The black diamond shows the point on the yield/ion temperature curve where the $\alpha$ deposited energy equals the bremsstrahlung and electron conduction losses. For the best HDC shot to-date, the hot spot $\rho r$ is high enough at moderate temperature ($\sim$4.7 keV) that the $\alpha$ deposited energy clearly exceeds the conduction losses leading to equilibrium as the ion temperature increases. At $\rho r < 0.18$ g/cm$^2$, the $\alpha$ deposited energy is never enough to compensate for the Bremsstrahlung and electron conduction losses.

Figure 4 shows most of the cryogenic DT layered implosions carried out on the NIF since the beginning of the National Ignition Campaign [31]. The performance of HDC implosions was improved first by increasing the implosion velocity and secondly by increasing the target scale and shortening the time between the end of the laser pulse and the time of peak neutron emission. At scale 5.75, increasing the implosion velocity resulted in higher ion temperature and thus implosion performance with a velocity scaling consistent with scaling of previous High Foot experiments [32]. At scale 5.75, 20 kJ of fusion yield was achieved using ”only” 1.1 MJ of laser light.
record fusions yields and hot spot $\rho$ demonstrated to increase stagnation pressure and yield (coast time) (figure 1-c). A reduced coast time was between the end of the laser and the peak neutron emission (coast time) (figure 1-c). A reduced coast time was demonstrated to increase stagnation pressure and yield for High Foot implosions [33]. These modifications led to record fusions yields and hot spot $\rho$ shown on figure 4 (red dots).

Experimental yield data are 30 to 50% of axisymmetric (2D) HYDRA [34] radiation hydrodynamic simulations including $\alpha$ deposition. The main source of yield degradation for these implosions is believed to be induced by the fill tube. The fill tube can inject mix and create local $\rho$ distortion of the shell. To bring simulations in closer agreement with experimental data, mix (i.e. carbon with 0.33% atomic fraction of tungsten) can be uniformly injected in a capsule-only calculation. Typically 50-100 ng of injected material is needed to reproduce the measured implosion performances.

The HDC campaign has produced, for the first time, a fusion energy (54 kJ) twice the peak kinetic energy of the implooding shell (21kJ). The implosion performance was improved from previous campaigns on the NIF by lowering X-ray drive asymmetry on the implooding shell. A hot spot areal density of 0.3 g/cm$^2$ was achieved, high enough to stop $\sim$ 85% of the $\alpha$ particles. The conditions in the hot spot with stagnation pressures of $\sim$360 Gbar, greater than the solar core pressure, is attracting a new community of scientists studying nucleosynthesis in the hot spot with stagnation pressures of $\sim$360 Gbar, greater than the solar core pressure, is attracting a new community of scientists studying nucleosynthesis on the NIF. Future shots in the campaign will aim at increasing the hot spot ion temperature by increasing the implosion velocity and further improving the shell areal density to improve the hot spot confinement.

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[13] Note1, the neutron fluxes reported in the letter would allow the measurement of neutron capture by unstable nuclides. Tm-171, due to its importance as a branching point nucleus in the slow process (s-process), has been proposed for an experiment on the NIF. Its relatively short half-life renders measurements at accelerator-based neutron irradiation facilities (such as LANSCE) extremely challenging. Adding 10 ng of TM-171 in the HDC ablator would produce $\sim$ 10$^7$ atoms of Tm172 that could be captured by the NIF Solid Radiochemistry Diagnostics (SRD).


