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# Measurement of the $^{169}\text{Tm}(n,3n)^{167}\text{Tm}$ cross section and the branching ratios in the decay of $^{167}\text{Tm}$

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The cross section for the  $^{169}\text{Tm}(n,3n)^{167}\text{Tm}$  reaction has been measured from 17 to 22 MeV using quasi-monoenergetic neutrons produced by the  $^2\text{H}(d,n)^3\text{He}$  reaction. This energy range was studied to resolve the discrepancy between previous  $(n,3n)$  cross-section measurements. In addition, the absolute  $\gamma$ -ray branching ratios following the electron-capture decay of  $^{167}\text{Tm}$  were measured. These results provide more reliable nuclear data for an important diagnostic that is used at the National Ignition Facility to estimate the yield of reaction-in-flight neutrons produced via the inertial-confinement-fusion plasma in deuterium-tritium capsules.

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## I. INTRODUCTION

The National Ignition Facility (NIF) at Lawrence Livermore National Laboratory is designed to drive deuterium-tritium (DT) inertial-confinement-fusion targets to ignition. As a result of DT fusion, neutrons are produced via the  $^3\text{H}(d,n)^4\text{He}$  reaction with energies  $E_n=14.1$  MeV. Higher-energy neutrons can be created when a D or T ion brings kinetic energy gained from elastic scattering with a fusion product in the plasma to the DT reaction. This multistep process produces so-called reaction-in-flight (RIF) neutrons with energies up to 30 MeV. Although the RIF neutrons represent less than 0.3% of the total neutron fluence, their spectrum carries important information about the fusion burn and charged-particle stopping powers in a warm, dense plasma [1–3].

A representative NIF neutron-energy spectrum [3] from DT fusion is shown in Fig. 1. The 15-MeV threshold for the  $^{169}\text{Tm}(n,3n)^{167}\text{Tm}$  reaction makes thulium a useful diagnostic for measurement of the RIF neutrons in the presence of copious 14-MeV neutrons.  $^{167}\text{Tm}$  also has convenient production and decay characteristics: a large  $(n,3n)$  cross section, a convenient half-life ( $t_{1/2} = 9.2$  days), and an intense  $\gamma$ -ray transition (with energy  $E_\gamma = 207.8$  keV and  $\gamma$ -ray emission branching ratio  $I_\gamma = (42 \pm 8)\%$ ). Since  $^{168}\text{Tm}$  is also produced from stable  $^{169}\text{Tm}$  through the  $(n,2n)$  reaction by 14.1-MeV neutrons, the ratio of  $^{167}\text{Tm}$  to  $^{168}\text{Tm}$  produced in a thulium monitor foil can provide a measure of the RIF neutron yield. The activation of foils at NIF and the subsequent offline counting using a Compton-suppressed high-purity germanium (HPGe) clover system to suppress backgrounds from the decay of  $^{168}\text{Tm}$  has been successfully used to measure  $^{167}\text{Tm}$  production and there-

fore determine the yield of RIF neutrons [3].

In Fig. 2, the previously published  $^{169}\text{Tm}(n,3n)$  cross-section measurements are shown [4–6]. In Ref. [5], the  $^{169}\text{Tm}(n,3n)$  reaction was measured using a large tank of gadolinium-loaded liquid scintillator to detect the emitted neutrons. In Ref. [6], the reaction was determined by using a NaI(Tl) well detector, while the neutron fluence was determined by proton-recoil telescope measurements. As can be seen, the experimental data for the  $^{169}\text{Tm}(n,3n)$  cross section is discrepant at energies above 19 MeV. Because the production of  $^{167}\text{Tm}$  depends on both the RIF neutron-energy spectrum, which is dropping off above 15 MeV, and the  $^{169}\text{Tm}(n,3n)$  cross section, which is increasing from 15–25 MeV, the largest contribution to  $^{167}\text{Tm}$  production from a typical NIF shot is due to the neutrons near 20 MeV in energy. Therefore, the 15–20% disagreement between the previous  $(n,3n)$  cross-section results in this energy region is concerning.

The easiest way to identify the presence of  $^{167}\text{Tm}$  in an activation foil is by detection of the 207.8-keV  $\gamma$  ray emitted following the electron-capture (EC) decay of the nucleus. However, to quantify the yield of  $^{167}\text{Tm}$ , the branching ratio of this  $\gamma$  ray must be accurately known. A current limitation is that this branching ratio, although large, had previously only been measured with a fractional precision of 19% [7]. The other  $\gamma$  rays emitted in the decay, such as the 531.5-keV  $\gamma$  ray with  $I_\gamma = (1.61 \pm 0.22)\%$ , have significantly smaller branching ratios which make it difficult to collect sufficient statistics to identify a peak especially with the large background from  $^{168}\text{Tm}$  in an activated sample.

In this work, two independent experiments were conducted to improve the nuclear data for the  $^{169}\text{Tm}(n,3n)^{167}\text{Tm}$  reaction cross section and the branching ratio of the 207.8-keV  $\gamma$ -ray to allow a better deter-

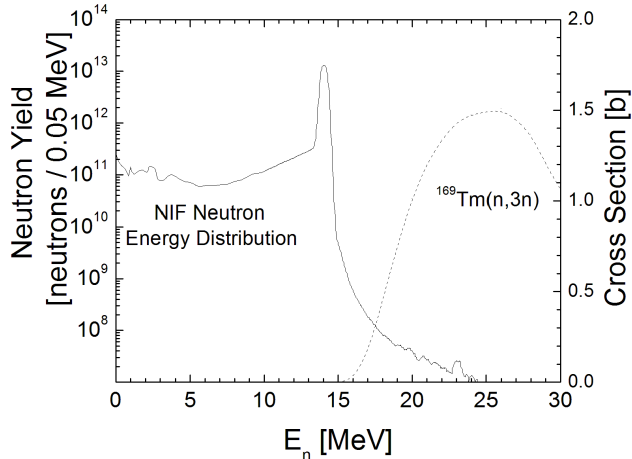


FIG. 1: Overlay of a simulated NIF neutron-energy distribution (y axis on left side) and the  $^{169}\text{Tm}(n,3n)^{167}\text{Tm}$  cross section (y axis on right side). Only reaction-in-flight neutrons have enough energy to produce  $^{167}\text{Tm}$ .

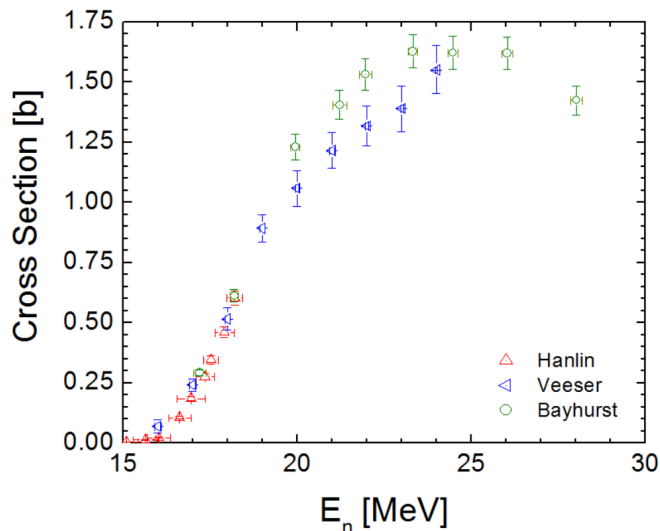


FIG. 2: (color online)  $^{169}\text{Tm}(n,3n)$  cross section data [4–6] from the EXFOR database.

mination of the number of RIF neutrons, which in turn provides valuable information for understanding the burn characteristics of a NIF plasma. The branching ratio for the 207.8-keV  $\gamma$  ray following  $^{167}\text{Tm}$  EC decay was measured precisely using facilities at Lawrence Berkeley National Laboratory (LBNL) and the University of California at Berkeley. With this improved branching ratio, a subsequent experiment was conducted to measure the  $^{169}\text{Tm}(n,3n)^{167}\text{Tm}$  cross section using the 10-MV Van de Graaff accelerator at the Triangle Universities Nuclear Laboratory (TUNL).

TABLE I: Activation samples for branching-ratio and cross-section measurements.

Target	Purity [%]	Thickness [ $\mu\text{m}$ ]
$^{165}\text{Ho}^*$	99.9	32(3)
$^{169}\text{Tm}^*$	99.9	261(4)
$^{nat}\text{Ni}$	99.994	514(8)
$^{nat}\text{Zr}$ (2.02% Hf)	97.91	241(4)

\* Monoisotopic

## II. $^{167}\text{Tm}$ BRANCHING-RATIO MEASUREMENT

### A. Experiment

A 32- $\mu\text{m}$ -thick  $^{165}\text{Ho}$  foil was irradiated using an  $\alpha$ -particle beam at the 88-Inch Cyclotron at LBNL to induce the  $^{165}\text{Ho}(\alpha,2n)^{167}\text{Tm}$  reaction. In addition to  $^{167}\text{Tm}$  ( $t_{1/2}=9.25$  d),  $^{168}\text{Tm}$  ( $t_{1/2}=93.1$  d) and  $^{166}\text{Tm}$  ( $t_{1/2}=7.70$  h) isotopes were produced by the  $^{165}\text{Ho}(\alpha,n)$  and  $^{165}\text{Ho}(\alpha,3n)$  reactions, respectively. With its shorter half life,  $^{166}\text{Tm}$  could be allowed to decay to stable  $^{166}\text{Er}$  prior to measurement. An  $\alpha$ -beam energy of 29 MeV was selected so that the induced activity of  $^{168}\text{Tm}$  was initially less than 0.5% of that of the desired  $^{167}\text{Tm}$ . Foil thickness was determined from the area and mass, and verified with calipers and  $\gamma$ -ray attenuation measurements. The  $^{165}\text{Ho}$  foil properties are summarized in Table I.

The x rays and  $\gamma$  rays from the activated foil were counted using both coaxial and planar HPGe detectors. Efficiencies for each detector were determined using  $^{57}\text{Co}$ ,  $^{133}\text{Ba}$ ,  $^{137}\text{Cs}$ ,  $^{152}\text{Eu}$ , and  $^{241}\text{Am}$  calibration sources. The calibration and foil counting was performed at a distance of 25 cm to minimize geometry uncertainties and avoid true-coincidence summing. The finer resolution and flatter efficiency curve of the planar HPGe detector in the lower-energy region was preferable for measuring the x rays. A sample spectrum measured with the planar HPGe detector is shown in Fig. 3. Self attenuation of the emitted x rays and  $\gamma$  rays in the foil was calculated using the foil thickness and photon cross sections [8, 9]. These corrections were less than 6% for the x rays, and were much smaller for the higher-energy  $\gamma$  rays.

### B. Results

The 49.128-keV  $K_{\alpha 1}$  x ray emitted from  $^{167}\text{Er}$  following  $^{167}\text{Tm}$  EC was used to determine the activity of  $^{167}\text{Tm}$  in the sample. The  $\gamma$ -ray branching ratios,  $I_\gamma$ , were then determined from

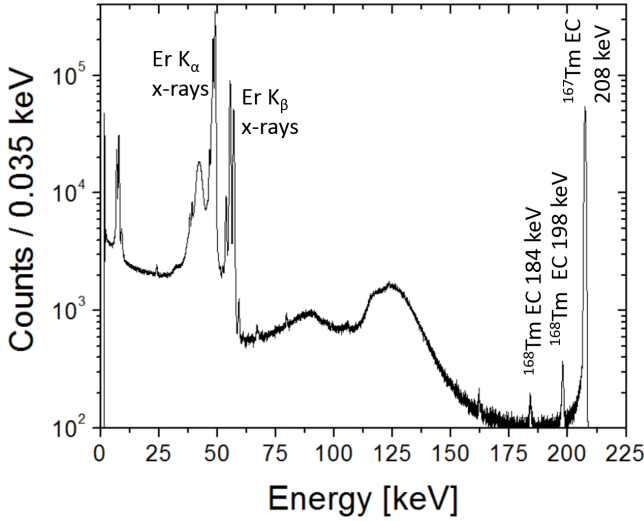


FIG. 3: Energy spectrum of the irradiated holmium foil measured using a planar HPGe detector.

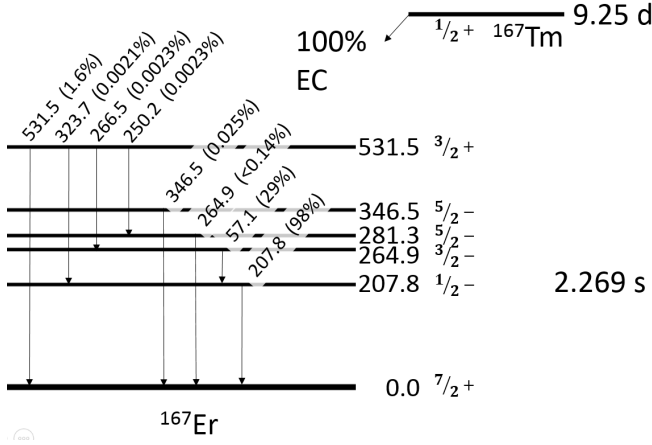


FIG. 4: Relevant decay scheme properties for the electron-capture decay of  $^{167}\text{Tm}$ . All energies are given in keV [10].

$$I_\gamma = \frac{\epsilon_x a_x}{\epsilon_\gamma a_\gamma} \frac{N_\gamma P_{K_{\alpha 1}}}{N_x}, \quad (1)$$

where  $\epsilon$  is the photopeak efficiency,  $a$  is the self-attenuation factor, and  $N$  is the number of counts in the peak of interest for the measured x rays and  $\gamma$  rays denoted by the subscripts x and  $\gamma$ , respectively, after correcting for x-ray contributions from the decay of  $^{168}\text{Tm}$  as described below.  $P_{K_{\alpha 1}}$  is the probability of  $K_{\alpha 1}$  x-ray emission following the EC decay. The  $K_\beta$  x rays were not used here due to the interference with the 57.1-keV  $\gamma$  ray following  $^{167}\text{Tm}$  decay.

The value of  $P_{K_{\alpha 1}}$  was calculated from the product of the total number of K-shell vacancies per decay and the probability of that vacancy resulting in the emission of a  $K_{\alpha 1}$  x ray. The K-shell vacancies may be caused by

the EC transition as well as any subsequent internally-converted transitions as  $^{167}\text{Er}$  de-excites to the ground state. The number of K-shell vacancies per decay from the EC transitions was calculated to be 0.8096(13) using values from Ref. [11] and the existing decay scheme [10], summarized in Fig. 4. The additional K-shell vacancies due to emission of internal conversion electrons from the excited daughter nucleus, primarily from the E3 transition of the first excited state at 207.8 keV, was calculated to be 0.1970(34) per decay using the BrIcc database for theoretical internal conversion coefficients [12]. This probability of creating a K-shell vacancy was then multiplied by the probability of  $K_{\alpha 1}$  x-ray emission, which is 47.5(10) per 100 K-shell vacancies from Table 7A of Ref. [13]. A value of  $P_{K_{\alpha 1}} = 0.478(10)$  was obtained, with an uncertainty dominated by the uncertainty in the  $K_{\alpha 1}$  x-ray intensity per K-shell vacancy.

The correction for the number of x rays emitted from the EC decay of the longer-lived  $^{168}\text{Tm}$  was calculated in a similar manner, with the activity determined from the measured 184-keV and 198-keV  $\gamma$ -ray peaks associated with this decay, as the  $\gamma$ -ray branching ratios are precisely known. This correction was only 0.4% (with negligible uncertainty) because of the small amount of  $^{168}\text{Tm}$  activity in the sample.

The results and uncertainties are summarized in Table II. The 207.8-keV and 531.5-keV  $\gamma$ -ray branching ratios were measured to be 0.419(16) and 0.0173(7), respectively. The branching ratios were determined with a relative uncertainty of about 3.8%, which is a factor of five better than the previous measurement [10].

As noted in Refs. [10, 14], the branching ratio of the 207.8-keV transition can also be calculated by decay-scheme normalization, if it is assumed that the second-forbidden EC transition to the ground state is negligible. In this case, after correcting for the internal-conversion contribution to each observed  $\gamma$  ray, the sum of the intensities of all the transitions to the ground state must equal 100%. This calculation produces a branching ratio of 0.416(7) [10], which agrees well with our experimental determination of the branching ratio.

### III. NEUTRON-ACTIVATION CROSS-SECTION MEASUREMENTS

#### A. Experiment

Quasi-mono-energetic neutrons were produced by the  $^2\text{H}(d,n)^3\text{He}$  reaction by bombarding a 3.0-cm-long deuterium gas cell [15] held at a pressure of 5.0 atm with deuterons accelerated by the High-Voltage FN Tandem Van de Graaff accelerator at TUNL. A stack of 0.9525-cm-diameter foil discs of  $^{169}\text{Tm}$ ,  $^{nat}\text{Ni}$ , and  $^{nat}\text{Zr}$  with properties listed in Table I was placed 0.9 cm from the end of the gas cell. Over five experimental runs, deuterons with energies  $E_D$  of 15.0 to 19.2 MeV were used to produce mono-energetic neutrons with energies

TABLE II: Branching-ratio results for  $^{167}\text{Tm}$ . The measured branching ratios for the 207.8-keV and 531.5-keV  $\gamma$ -rays are provided in the last row, with their absolute uncertainties given in parentheses. The primary contributors to these uncertainties are listed at the top of the table, given in relative %.

Primary sources of uncertainty			
	$K_{\alpha 1}$ x ray	207.8 keV	531.5 keV
Counting statistics	0.5%	0.7%	0.4%
Self attenuation in foils	1.1%	0.04%	0.01%
HPGe detector efficiency	2.1%	2.1%	1.8%
$P_{K_{\alpha 1}}$ x ray	2.1%		
Measured absolute BR for $^{167}\text{Tm}$ per 100 EC decays			
		41.9(16)	1.73(7)

of 17.5 to 21.5 MeV along the beam axis.

The energy distribution of these neutrons impinging on the foil is broadened by two effects. First, the reacting deuterons have an energy spread caused by energy-loss effects in the 6.5- $\mu\text{m}$  Havar entrance foil and across the gas cell. These effects were determined by a Monte Carlo method using the Stopping Range in Matter (SRIM) program [16]. Second, with such a small distance between the gas cell and the activation foils, neutrons emitted off the beam axis can strike the foil. Because of the kinematics, the off-axis neutrons have a lower energy than the neutrons emitted parallel to the deuteron beam. This kinematic effect was taken into account using the differential cross section of the  $^2\text{H}(\text{d},\text{n})^3\text{He}$  reaction [17] and assuming uniform production along the length of the gas cell. As an example, for 21.5-MeV neutrons produced at the center of the gas cell, the energy distribution of the neutrons hitting the foils is shown in Figure 5. The spread in neutron energies at other settings is similar in shape. A summary of the average energies of the neutrons in the mono-energetic peak and their energy spread is presented in Table III.

Deuteron breakup contributes additional neutrons with lower energies from reactions in the deuterium gas cell, entrance collimator, Havar entrance foil, and tantalum beam stop. The neutron-energy distribution was determined at each energy setting so that the activation by lower-energy neutrons could be corrected. The neutron beam was pulsed and the neutron time-of-flight (NTOF) was measured with a 3.81-cm-diameter  $\times$  3.81-cm-long cylindrical liquid scintillation (BC501A) neutron detector placed along the beam axis at a distance of 287.5 cm from the end of the gas cell. The measured NTOF distributions (Fig. 6a) were converted to neutron energy spectra (Fig. 6b) using relativistic kinematics. The finite time resolution of the pulsing system, as determined by the width of the  $\gamma$ -ray flash from deuterons on the collimator, dominates the observed width of the quasi-monoenergetic peak. The energy of the quasi-monoenergetic neutron peaks measured in this way were consistent with the estimates based on the Tandem set-

TABLE III: Deuteron energies ( $E_D$ ) for the five experimental runs at TUNL and the corresponding neutron energies ( $E_n$ ). The  $E_n$  on axis and the average value of  $E_n$  are listed. The energy spread of the neutrons striking the foils is due primarily to the energy spread of the reacting deuterons and the angular dependence of the neutron energy distribution in the  $^2\text{H}(\text{d},\text{n})^3\text{He}$  reaction.

$E_D$ [MeV]	On axis	Average	Energy Spread
	$E_n$ [MeV] <sup>a</sup>	$E_n$ [MeV] <sup>b</sup>	FWHM [MeV]
15.02	17.5	17.35	0.18
16.06	18.5	18.34	0.18
17.12	19.5	19.33	0.19
18.12	20.5	20.32	0.19
19.22	21.5	21.31	0.20

<sup>a</sup> Neutron energy calculated for reactions at the center of the gas cell based on the magnet settings <sup>b</sup> Average energy of neutrons striking the activation foils

tings. The neutron detection threshold was determined using  $^{22}\text{Na}$ ,  $^{88}\text{Y}$ , and  $^{241}\text{Am}$ -Be sources and the resulting neutron detection efficiency was calculated with the NEFF7 code [18] as described in Ref. [19]. Only the relative neutron efficiency was required for the corrections applied.

At these deuteron energies, the breakup is significant, as can be seen in Figure 6. In order to correct for activation due to lower-energy neutrons, Eq. 2 was used to determine the fractional contribution (FC) of the activity induced due to the breakup neutrons relative to the total activity induced [20],

$$FC = \frac{\int_0^{E_c} \phi(E)\sigma(E)dE}{\int_0^{E_c} \phi(E)\sigma(E)dE + \phi_x\sigma_x}. \quad (2)$$

In Eq. 2,  $\phi(E)$  is the neutron energy distribution determined by the NTOF measurements,  $\sigma(E)$  is the cross section for the reaction of interest,  $E_c$  is the cut-off energy taken as the minimum point in the valley between the neutron peak and the deuteron breakup,  $\phi_x$  is the integration of the neutron peak, and  $\sigma_x$  is the cross section at the peak energy. This factor was determined for each reaction of interest using excitation functions from nuclear data evaluations.

In all cases, only the shape of the cross section impacted the correction. The calculated FC values were more sensitive to uncertainties in the neutron-energy distribution than to differences between evaluated cross sections. For  $\text{Tm}(\text{n},\text{Xn})$  reactions, the ENDF/B-VII.1 evaluation was used. For the reactions that do not have ENDF evaluations above 20 MeV, other sources were used. Data for the  $^{90}\text{Zr}(\text{n},2\text{n})$  reaction was taken from the International Nuclear Data Committee evaluation [21], while for the  $^{94}\text{Zr}(\text{n},\alpha)$  reaction, the JEFF-3.2 evaluation [22] was used and for the  $^{58}\text{Ni}(\text{n},2\text{n})$  reaction, the IRDFF-1.05 evaluation [22] was used. Uncertainty in FC includes the uncertainty from the excitation curve and the mea-

TABLE IV: Flux correction (FC) factors calculated by Eq. 2 to account for activation by lower-energy neutrons produced from deuteron breakup.

Reaction	$^{90}\text{Zr}(n,2n)^{89}\text{Zr}$	$^{169}\text{Tm}(n,2n)^{168}\text{Tm}$	$^{169}\text{Tm}(n,3n)^{167}\text{Tm}$	$^{58}\text{Ni}(n,2n)^{57}\text{Ni}(10)$	$^{94}\text{Zr}(n,\alpha)^{91}\text{Sr}$
Energy [MeV]	FC	FC	FC	FC	FC
17.35(10)	0.0523(26)	0.394(20)	0.00593(30)	0.0422(21)	0.118(6)
18.34(10)	0.0735(37)	0.609(30)	0.0108(5)	0.0590(3)	0.189(9)
19.33(10)	0.122(6)	0.765(38)	0.0224(11)	0.0950(5)	0.301(15)
20.32(10)	0.210(11)	0.856(21)	0.0424(21)	0.149(7)	0.438(22)
21.31(10)	0.440(22)	0.924(46)	0.117(6)	0.331(16)	0.645(32)

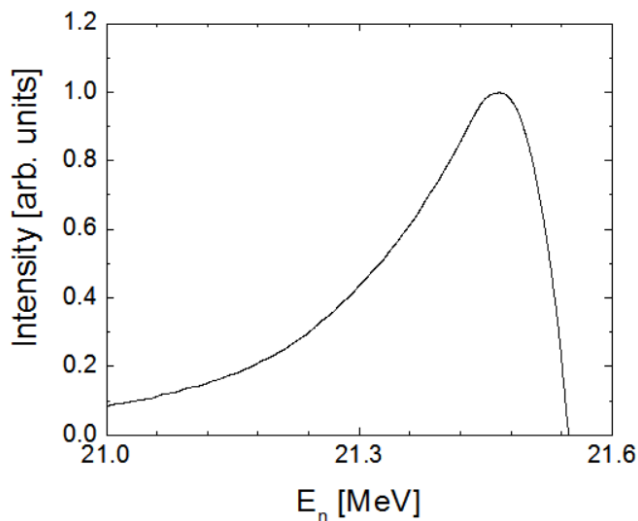


FIG. 5: Calculated energy distribution of neutrons striking the foil at the 21.5-MeV settings. The spread in neutron energies is primarily caused by kinematic effects due to the extended geometry of the foil.

sured NTOF distribution propagated through Eq. 2. The calculated flux corrections for each energy and isotope are shown in Table IV.

The activation foils were counted on coaxial HPGe detectors at TUNL immediately following neutron irradiation. The foils were subsequently shipped to UC Berkeley for further counting. The primary  $\gamma$ -ray lines used for identifying isotopes are listed in Table V. Corrections were made for the minor variations in beam current during activation [23] and the decay of the sample between production and measurement using the half-lives in Table V. The uncertainties in these decay corrections are less than 1% because all the half-lives are precisely known. The neutron attenuation through the foils was calculated to be negligible, however self attenuation of the emitted  $\gamma$  rays in the foils was determined and used for subsequent corrections. All detectors were calibrated using standard sources. In both locations, samples were counted at a distance of 10 cm. Summing effects were determined using a GEANT4 simulation [24] of the detector and its surroundings. The corrections required were minimal, and the most significant were  $\sim 2\%$ .

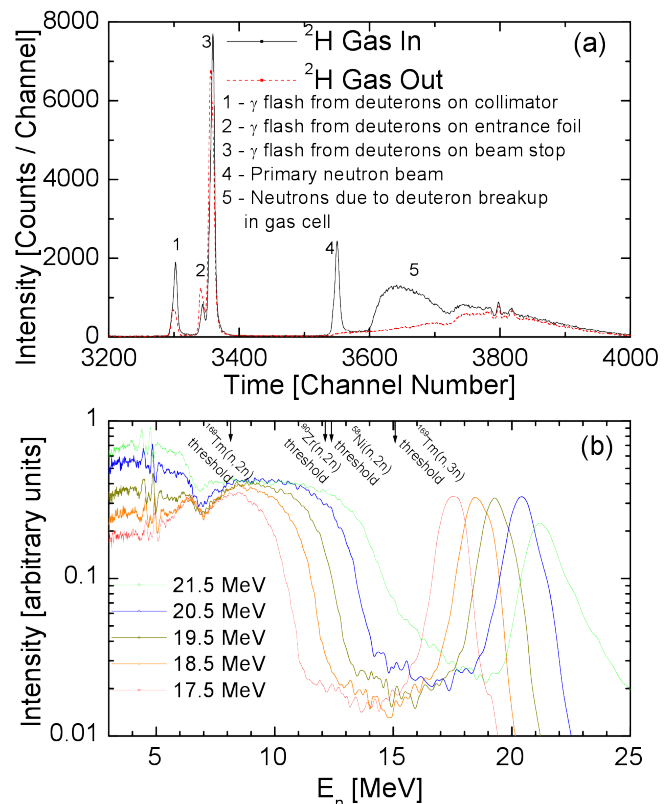


FIG. 6: (color online) (a) Time-of-flight spectrum for the settings used to produce 20.5-MeV neutrons. Additional measurements were performed with  $^2\text{H}$  removed from gas cell to help identify features noted in the plot. (b) Neutron-energy spectra determined from time-of-flight measurements. The spectra are normalized by area in the quasi-monoenergetic peak and are shown with equal energy bin widths. The time-of-flight measurement at the 21.5-MeV settings was performed under timing conditions that resulted in greater broadening of the highest-energy neutrons.

## B. Results

The neutron flux was determined from the  $^{90}\text{Zr}(n,2n)^{89}\text{Zr}$  monitor reaction using the cross section from Ref. [21], which had uncertainties which varied between 1.44% to 3.46% in the energy range studied.

TABLE V: Decay data for nuclear reactions used in this work [22].

Reaction	Q-value [MeV]	Product half-life	$E_\gamma$ [keV]	$I_\gamma$
$^{90}\text{Zr}(n,2n)^{89}\text{Zr}$	-11.7	78.41(12) h	909.15(15)	0.9904(3)
$^{169}\text{Tm}(n,2n)^{168}\text{Tm}$	-8.03	93.1(2) d	198.251(2)	0.5449(16)
			815.989(5)	0.5095(16)
$^{169}\text{Tm}(n,3n)^{167}\text{Tm}$	-14.9	9.25(2) d	207.801(5)	0.419(16)
$^{58}\text{Ni}(n,2n)^{57}\text{Ni}$	-12.2	35.60(6) h	1377.63(3)	0.817(24)
$^{94}\text{Zr}(n,\alpha)^{91}\text{Sr}$	+2.03	9.63(5) h	1024.3(1)	0.335(11)

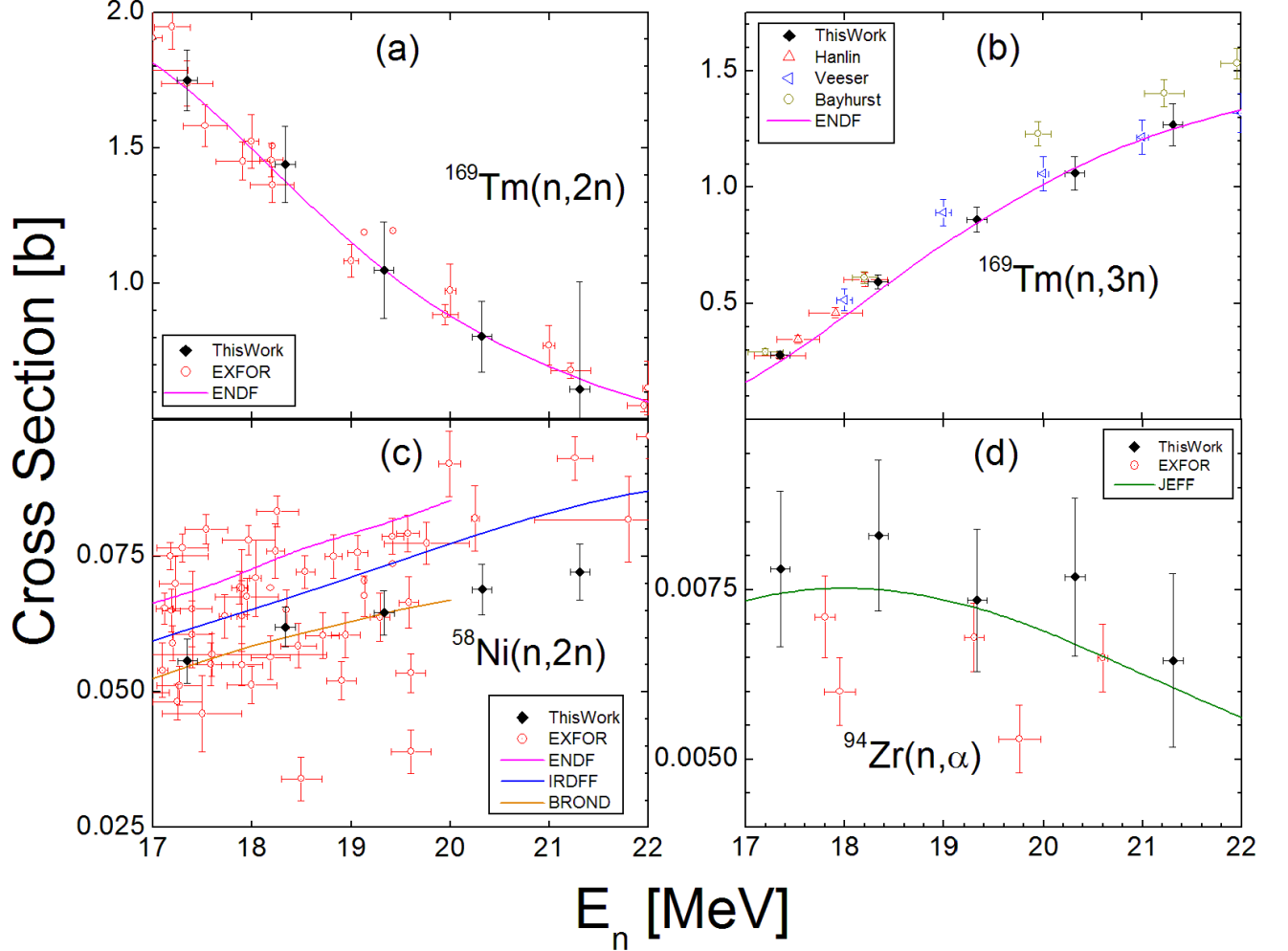


FIG. 7: (color online) Experimental results from this work compared with the previous data summarized in EXFOR and evaluations [22].

Cross sections were measured for the reactions listed in Table VI. For the  $^{169}\text{Tm}(n,2n)$  and  $^{94}\text{Zr}(n,\alpha)$  reactions, for which the mono-energetic neutron peak was much higher in energy than the reaction threshold, the flux correction (Table IV) was the largest contributor to uncertainty. The agreement of the present  $^{169}\text{Tm}(n,2n)$  cross-section results with previous experimental data [22] demonstrates that the NTOF measurements and lower-energy-neutron correction techniques are reliable. This

provides confidence in the smaller corrections applied for other reactions. Other significant sources of uncertainty were HPGe  $\gamma$ -ray detection efficiency (2–3%), and in some cases, the  $\gamma$ -ray branching ratios. Other factors in the calculation contribute uncertainties of less than 1%.

The detection of the 207.8-keV  $\gamma$  ray simplifies the measurement of  $^{167}\text{Tm}$  production and minimizes the assumptions required to interpret the data. The results for

$^{169}\text{Tm}(n,3n)^{167}\text{Tm}$ , support the ENDF evaluation and are in agreement with the data of Veeseer *et al* [5] as can be seen in Fig. 7b. These  $^{169}\text{Tm}(n,3n)$  cross-section results are about 15–20% lower than those from Ref. [6] at energies around 20 MeV.

Existing experimental cross-section data for the  $^{58}\text{Ni}(n,2n)^{57}\text{Ni}$  reaction is discrepant [22]. As shown in Fig. 7c, the uncertainty of the individual measurements is of the order of 10%, however, the literature values for this cross section span nearly a factor of two in the energy range 17–22 MeV. The present data agrees with the evaluation by Brond [22].

Results for  $^{94}\text{Zr}(n,\alpha)^{91}\text{Sr}$  are also reported for which only two other measurements have been made in this energy region [25, 26]. Although the  $(n,\alpha)$  cross section is much smaller than the  $(n,2n)$  and  $(n,3n)$  cross sections for  $^{94}\text{Zr}$ , it carries important additional information that constrains nuclear models used to determine optical model and energy level density parameters. Our  $^{94}\text{Zr}(n,\alpha)$  cross-section results support the JEFF 3.2 evaluation.

#### IV. SUMMARY

The motivation for this work was to solidify the nuclear data used to interpret an activation diagnostic that measures the yield of reaction-in-flight neutrons at NIF. The  $^{169}\text{Tm}(n,3n)^{167}\text{Tm}$  reaction cross section and the branching ratio of the 207.8-keV  $\gamma$  ray following the EC decay of  $^{167}\text{Tm}$  were remeasured.

The branching ratio was measured using the facilities at UC Berkeley and the 88-Inch Cyclotron at LBNL. A thin holmium foil was irradiated using a 29-MeV  $\alpha$ -particle beam inducing the  $^{165}\text{Ho}(\alpha,2n)^{167}\text{Tm}$  reaction. The foil was counted using well-calibrated coaxial and planar HPGe detectors. The branching ratios for the emission of 207.8-keV and 531.5-keV  $\gamma$  rays were determined with fractional uncertainties of approximately 3.8%. These uncertainties are five times smaller than the previous measured results [22].

Using this measured branching ratio for the 207.8-keV transition, a subsequent experiment was conducted to measure the  $^{169}\text{Tm}(n,3n)^{167}\text{Tm}$  cross section. Quasi-monoenergetic neutron beams were produced using the 10-MV Van de Graaff accelerator at TUNL. The cross section was measured with a precision of 5–8% in the 17–22 MeV energy range that is important for determining the RIF-neutron fluence at NIF. In addition, the cross sections for  $^{58}\text{Ni}(n,2n)$  and  $^{94}\text{Zr}(n,\alpha)$  were measured over the same energy range.

These branching-ratio and cross-section results provide reliable nuclear data which reduce the overall uncertainties on the measured reaction-in-flight yield at NIF to about 5–8% in an important energy region between 17 and 22 MeV. This work confirms the  $^{169}\text{Tm}(n,3n)^{167}\text{Tm}$  cross section ENDF evaluation and the Veeseer *et al.* [5] results and provides strong evidence that the experimen-

tal results of Bayhurst *et al.* [6] are 15–20% too large.

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TABLE VI: Measured cross sections.

Reaction	$^{169}\text{Tm}(n,2n)^{168}\text{Tm}$	$^{169}\text{Tm}(n,3n)^{167}\text{Tm}$	$^{58}\text{Ni}(n,2n)^{57}\text{Ni}$	$^{94}\text{Zr}(n,\alpha)^{91}\text{Sr}$
Energy [MeV]	$\sigma$ [b]	$\sigma$ [b]	$\sigma$ [b]	$\sigma$ [b]
17.35(10)	1.75(12)	0.279(15)	0.056(4)	0.0078(12)
18.34(10)	1.44(14)	0.593(31)	0.062(4)	0.0083(12)
19.33(10)	1.05(19)	0.861(54)	0.065(5)	0.0074(11)
20.32(10)	0.804(134)	1.06(7)	0.069(5)	0.0077(12)
21.31(10)	0.611(369)	1.27(10)	0.072(5)	0.0065(13)

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